Computational

Experimental

Masashi Shimada and Pattrick Calderoni - Idaho National Laboratory

The Safety and Tritium Applied Research (STAR) facility:

- DOE less than Hazard Category 3 radiological facility that handles:
 - Tritium (licensed for up to 1.6 gram)
 - Beryllium containing compounds (e.g. FLiBe)
 - Radioactive materials (< 1 mSv/hr = 100 mrem/hr at 30 cm)
- Experimental capabilities (see the fact sheet below)
 - Linear plasma device for FW and divertor material testing
 - Tritium Plasma Experiment (TPE)
 - Provides T retention and trapping, plasma-material interaction
 - Key experimental capabilities:
 - Tritium plasma (< 500 Ci / shot),
 - High-flux plasma (>10²² m⁻²s⁻¹),

Mixed (D-T-He) plasma

Material size: (6-25) mm OD & (0.25-2.0) mm thickness

~7cm

- Permeation systems to measure H/D/T mass transport properties
 - Tritium Gas Absorption Permeation (TGAP)
 - Provides diffusivity, solubility, and permeability of tritium (T)
 - Key experimental capabilities:
 - Tritium (T₂) gas (< 25 Ci per campaign)
 - Independent control of H₂ and T₂ partial pressures
 - Low tritium partial pressure $(10^{-12} < p_{T_2} [Pa] < 10^2)$
 - Low temperature (RT to 700 °C)
 - Static Gas Absorption Permeation (SGAP)
 - Provides diffusivity, solubility, and permeability of H & D
 - Key experimental capabilities:
 - H_2 and D_2 gas pressure (10³< $p_{D2}[Pa]$ < 10⁵), high temperature (RT to 950 °C)
- Forced-convection loops to advance tritium extraction efficiency
 - Tritium Extraction experiment (TEX)
 - Lead lithium (PbLi) loop to improve efficiency of T extraction
 - Test tubular membranes (Fe, V, Nb, coated tubes, etc.) of vacuum permeator •
 - Additional test section configurations possible.
 - Molten Salt Tritium Transport Experiment (MSTTE)
 - Molten salt (FLiNaK, FLiBe) loop to study T extraction methods
 - SALAMANDER Exhaust pumping technology development for compact T fuel cycle
 - Permeation Experiment for Asymmetric Surfaces (PEAS)
 - Advances material sciences (e.g. surface condition, material interdiffusion, coating material) required to maintain H isotopes (H,D) super-permeation for metal foil pump development
 - Gas-, Atomic-, and plasma-driven H/D permeation system with in-vacuo surface cleaning (with Ar ion sputter guns) and Auger surface characterization on both front/back surfaces
- Material irradiation capabilities for solid breeder materials
 - Neutron RADiography (NRAD) reactor 300 kW TRIGA research reactor
 - Provides cost-effective neutron irradiation option with low neutron flux

System-level T & safety analysis for paths to commercialization

- Fusion safety analysis by MELCOR-TMAP
 - MELCOR is a US NRC computer code developed by SNL to analyze a wide range of accidents and transients that may occur in a nuclear power plant to model and analyze the safety and performance of nuclear system.
 - MELCOR-fusion (developed by INL) was used for many fusion systems:
 - ITER's safety analysis; V&Vd for ITER Non-Site Specific Safety Report (NSSR) 1&2 in 1997, Generic Site Safety Report (GSSR) in 2001, and the Report Preliminary on Safety (RPrS) in 2011, IFMIF-DOMES etc.
 - Many of US design studies including, ARIES, FNSF, FESS studies in the US, and DEMO safety analysis in Europe and Japan,
 - MELCOR-TMAP added Tritium Migration Analysis Program into MELCOR
 - Trace element tracking for H, D, T, Surface reactions, vapor-liquid-solid equilibrium model, Composite interface model for solid structures implemented (coated surfaces), Non-condensable gas modeling improved, HTO modeling
- Reliability, Risk, and Optimization by RAVEN, EMRALD, and SAPHIRE
 - INL can provide the following safety, optimization, and regulatory supports:
 - Hazard analysis, Risk-informed design and system analysis, Uncertainty analysis for multi-physics modeling and simulation, Gather and analyze reliability database for fusion systems, Preliminary PRA and risk metric development, Dynamic risk/reliability assessments, Physics-based risk/reliability assessments, Probabilistic physics-of-failure modeling, Availability and design optimization, Regulatory analysis

Accelerating nuclear innovation through advanced ModSim

MOOSE (Multiphysics Object-Oriented Simulation Environment)

- Is a modular and complete platform for fission and fusion multiphysics
 - Multiphysics, open-source, massively parallel, NQA-1 Software Quality Assurance
 - MOOSE applications covers wide range of physics:, https://mooseframework.inl.gov/
 - Chemical Reactions, Contact, **Electromagnetics**, Fluid Properties, Fluid Structure Interaction, Function Expansion Tools, Geochemistry, Heat Conduction, Level Set, Navier Stokes, Peridynamics, Phase Field, Porous Flow, Ray Tracing/Particle Tracking, rDG,, Stochastic Tools, Tensor Mechanica Therma

MOOSE-based fusion applications:

Tritium Migration Analysis Program – TMAP8,

https://mooseframework.inl.gov/TMAP8/

Performs multiscale tritium transport and thermal analysis fusion systems

SALAMANDER (Software for Advanced Large-scale Analysis of MAgnetic confinement for Numerical Design, Engineering & Research) brings together different MOOSE-based physics module, including TMAP8 and Cardinal. https://mooseframework.inl.gov/salamander/

- Performs multiphysics analysis of FW & blanket systems
- Cardinal: MOOSE+OpenMC+NekRS,

https://cardinal.cels.anl.gov/

- Exascale simulation tool showcase for coupling non-MOOSE-based applications and uses MOOSE physics "glue", and has simulated:
 - Gas-cooled microreactors, sodium fast reactors, molten salt reactors, high-temperature gas reactors, and tritium breeder blankets
- MOOSE-based fusion applications developed by UKAEA
 - Aurora, Apollo, Proteus, Phaethon, etc...
- Other ModSim development under DOE NE for advanced reactors
 - Virtual Test Bed, https://mooseframework.inl.gov/virtual_test_bed/
 - Digital Innovation Center of Excellence (DICE), https://dice.inl.gov/

